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INSTITUT
DE RADIOPROTECTION
ET DE SÛRETÉ NUCLÉAIRE

Faire avancer la sûreté nucléaire

Use of integral experiments in support to the validation of JEFF-3.2 nuclear data evaluation

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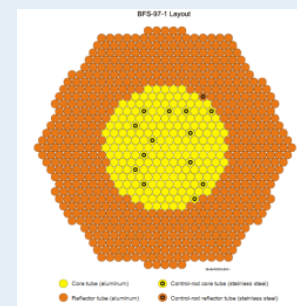
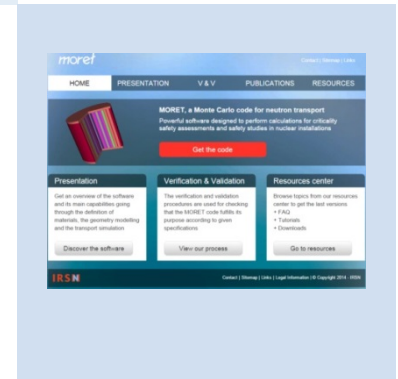
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Methodology



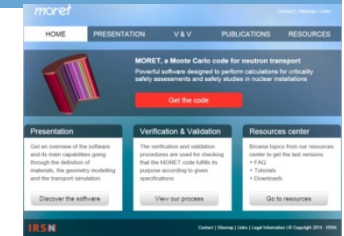
■ Selection of experimental cases in the ICSBEP Handbook

- Huge variety of configurations and neutron spectrum
- Quality of experimental data
- Selection according to experimental uncertainties

■ Selection of 238 cases covering:

- FAST energy range,
- INTERMEDIATE energy range,
- THERMAL energy range,
- SPECIAL isotopes (^{233}U , ^{237}Np ,...)
 - Privilege few cases per series and many independent series to avoid Correlations between cases and between series

Methodology



Calculations run with Monte Carlo Continuous energy code MORET 5.C.1

- Use of JEFF-3.1.1 and JEFF-3.2 evaluations for nuclear data
- Use of probability tables for unresolved energy range
- $\sigma_{MC} = 0.00020$

Comparison of MORET 5.C.1 k_{eff} results run with JEFF-3.1.1 and JEFF-3.2 with the benchmark k_{eff}

- If difference of k_{eff} exceeds $3\sigma_{comb}$ then a bias has been detected, otherwise no bias

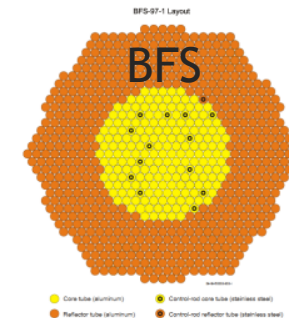
$$\text{with } \sigma_{comb} = \sqrt{\Delta k_{eff}^2 + \sigma_{MC}^2}$$

Selection of experiments



FAST energy range

- Uranium
 - $89.5 \text{ wt.}\% < {}^{235}\text{U} < 97.7 \text{ wt.}\%$
 - Reflectors: none, water, depleted uranium, CH_2 , alumina, tungsten
- Plutonium
 - $1.8 \text{ wt.}\% < {}^{240}\text{Pu} < 20.16 \text{ wt.}\%$
 - Reflectors: none, water, ${}^{\text{nat}}\text{U}$, CH_2 , alumina, tungsten



INTERMEDIATE energy range

- HEU powders: $93.3 \text{ wt.}\% {}^{235}\text{U}$
 - Various water content and interstitial moderators (Water, CH_2)
 - Reflector: 15.24 cm CH_2
- BFS experiments
 - Hexagonal lattice of tubes with stacked pellets of $\text{U}(\text{dep})\text{O}_2$, $\text{Pu}(95)$
 - Variable water content: CH_2 pellets and dowels,



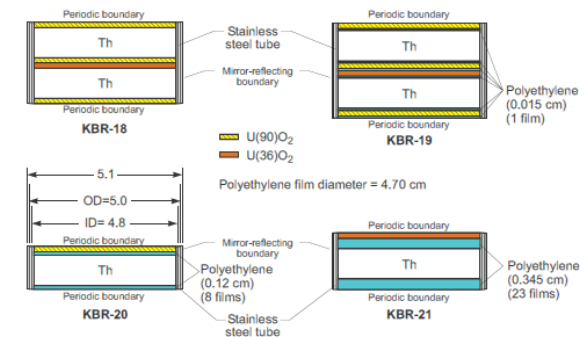
Selection of experiments (continued)

Intermediate energy range

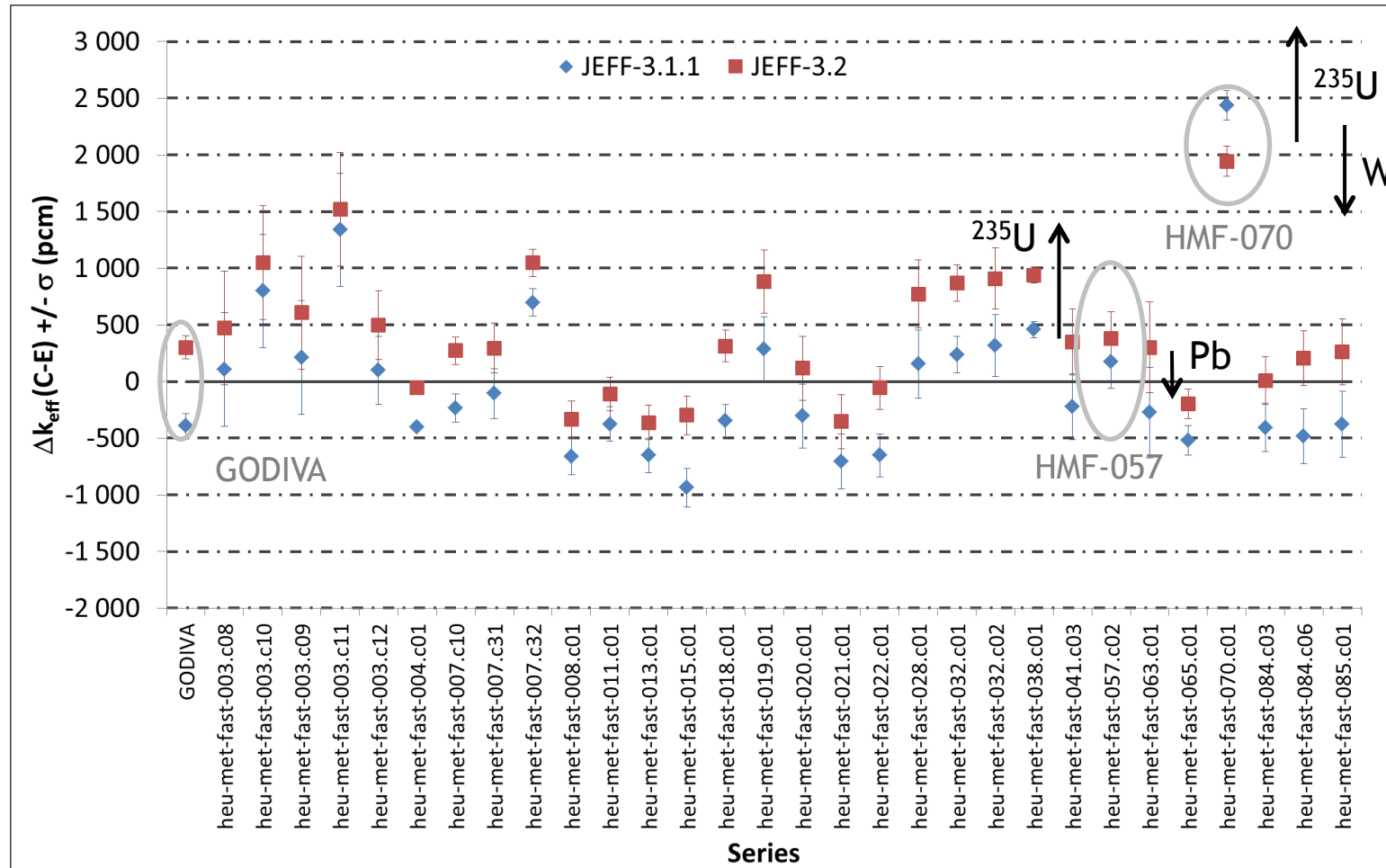
- ICI: Infinite lattice of stacked pellets
 - Intermediate and highly enriched uranium and thorium

THERMAL energy range

- Lattices of UO_2 rods in water
 - $2.6 \text{ wt.}\% < {}^{235}\text{U} < 6.9 \text{ wt.}\%$
 - Reflectors: water
- Lattices of UO_2 rods in water with absorber and SS, lead reflector
 - $2.6 \text{ wt.}\% < {}^{235}\text{U} < 6.9 \text{ wt.}\%$
 - Absorber: B, Cd, Gd...
 - Reflectors: lead, stainless steel, water
- Lattices of UO_2 - PuO_2 rods in water
 - $3 \text{ wt.}\% < \text{Pu}/\text{UPu} < 22 \text{ wt.}\%$
 - $8.5 \text{ wt.}\% < {}^{240}\text{Pu} < 22.2 \text{ wt.}\%$
 - $1.7 < V_{\text{mod}}/V_{\text{ox}} < 17.5$

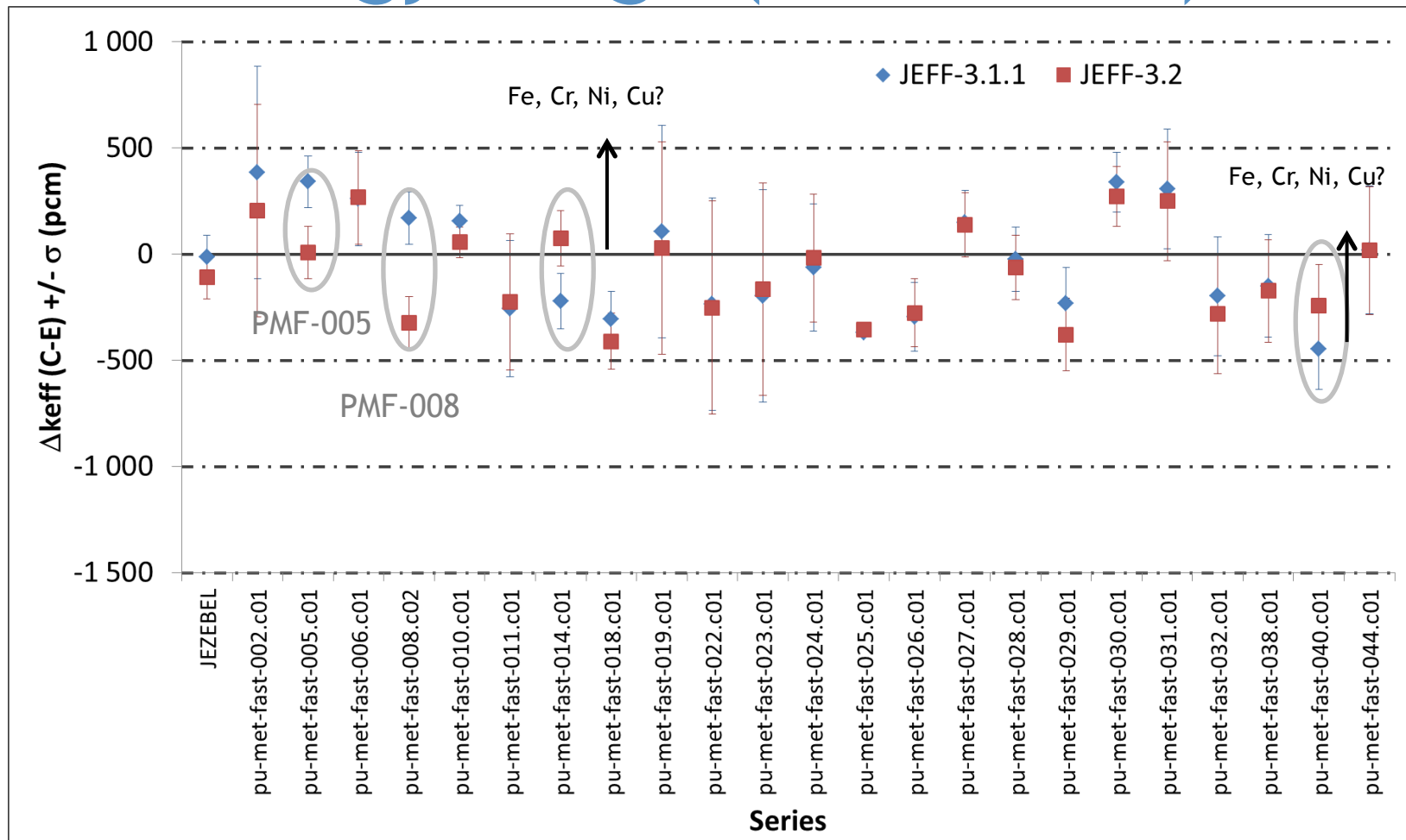


FAST energy range: ^{235}U



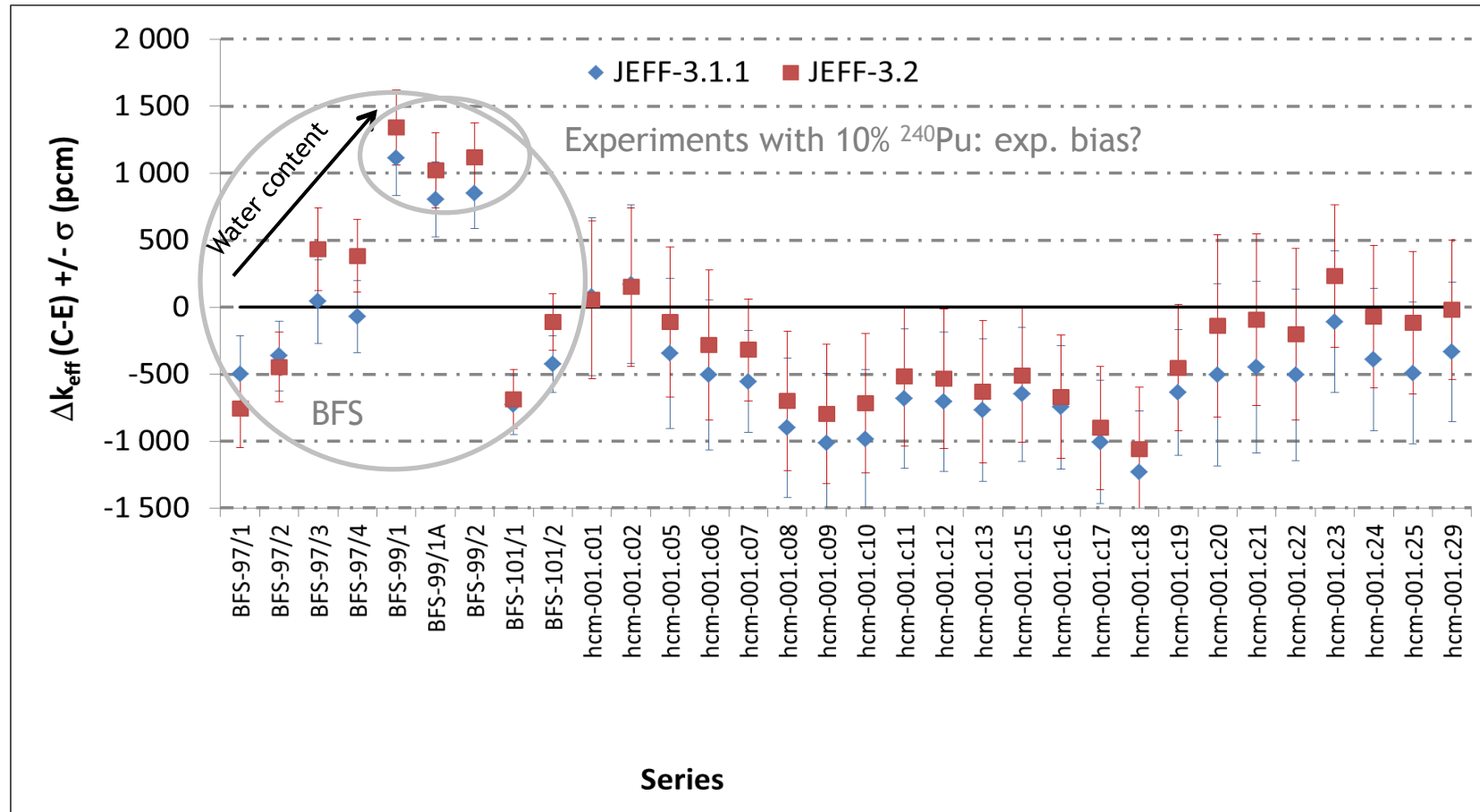
- ↗ Shift of +500 pcm of results, due to ^{235}U
- ↗ W (HMF-070) and Pb (HMF-057): compensation effects due to new evaluation

FAST energy range (continued): Pu



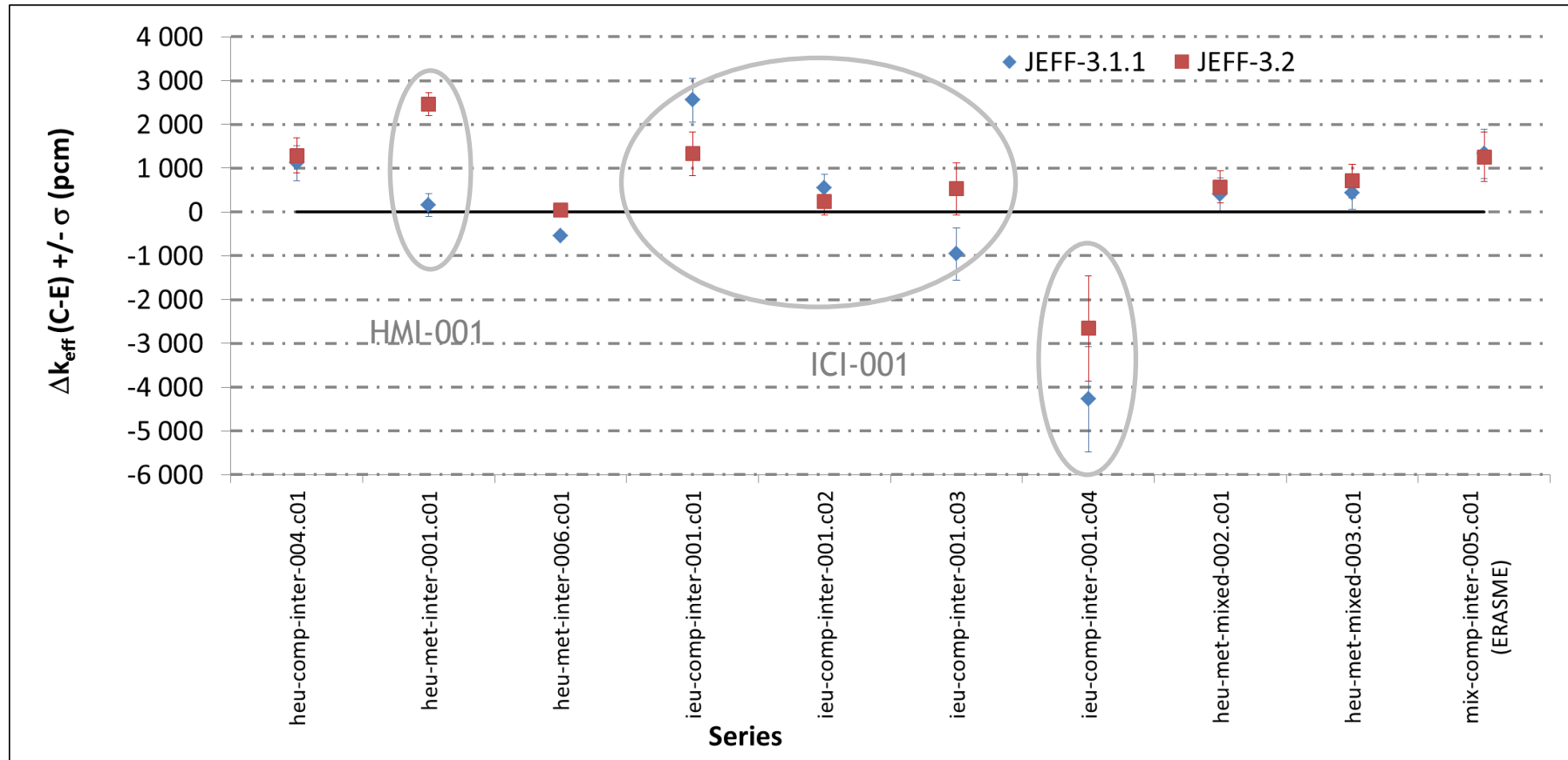
- ↗ No significant modification for JEZEBEL due to Pu
- ↗ No significant impact of ^{240}Pu
- ↗ Strong reduction of k_{eff} by -500 pcm with JEFF-3.2 for ^{232}Th (PMF-005, -008)

INTERMEDIATE energy range: U and Pu



- ↪ BFS: evolution of k_{eff} with JEFF-3.2 depending of neutron spectrum
- ↪ Tendency of k_{eff} results with water content still observed with JEFF-3.2
- ↪ HCM-001: slight improvement of results

INTERMEDIATE energy range: U and Pu



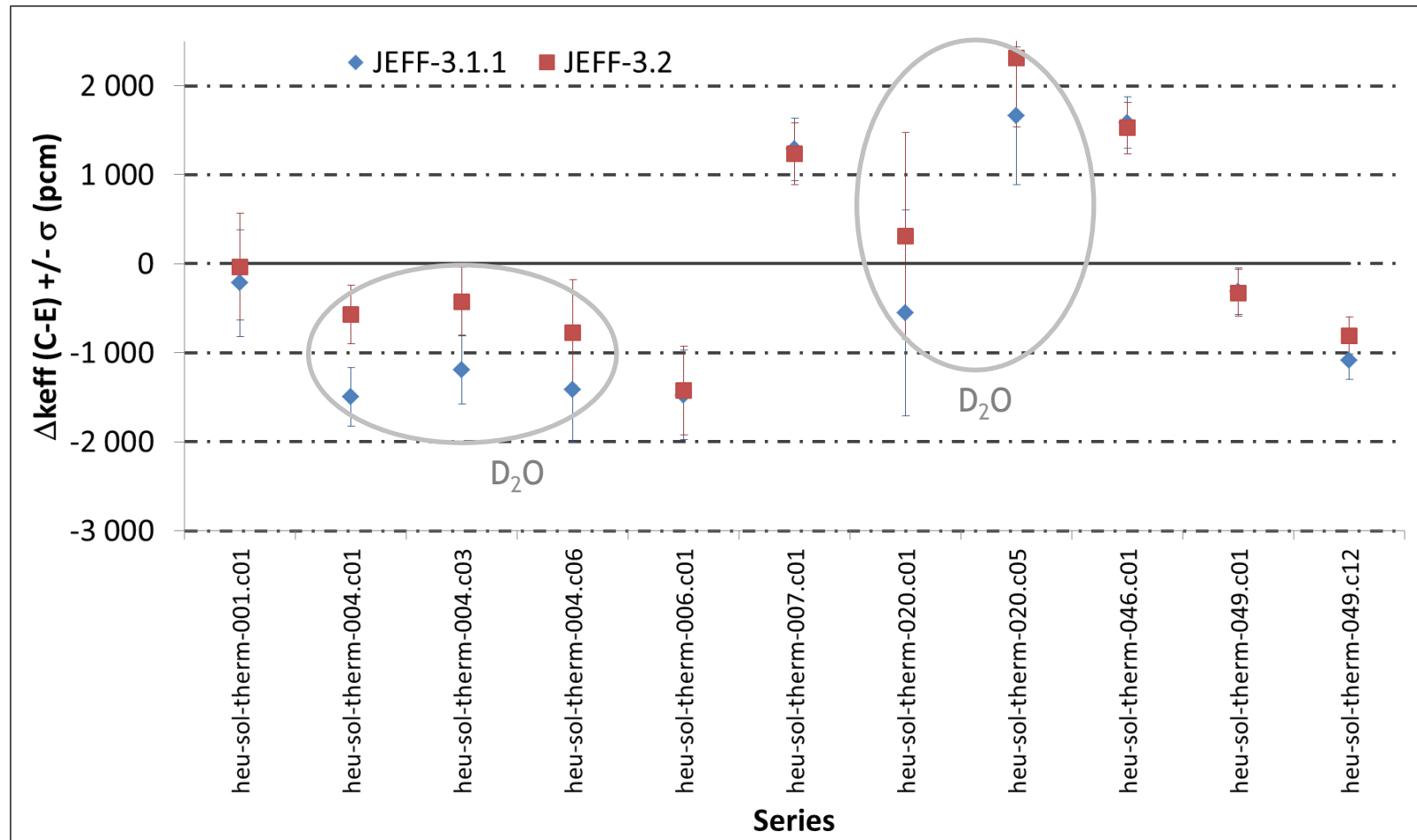
↗ For HML, increase of k_{eff} until 2500 pcm (HML-001) [?] Fe, Cr, Ni, Cu?

↗ ICI:

↗ Fast energy range: decrease by -300 to -1200 pcm (^{232}Th)

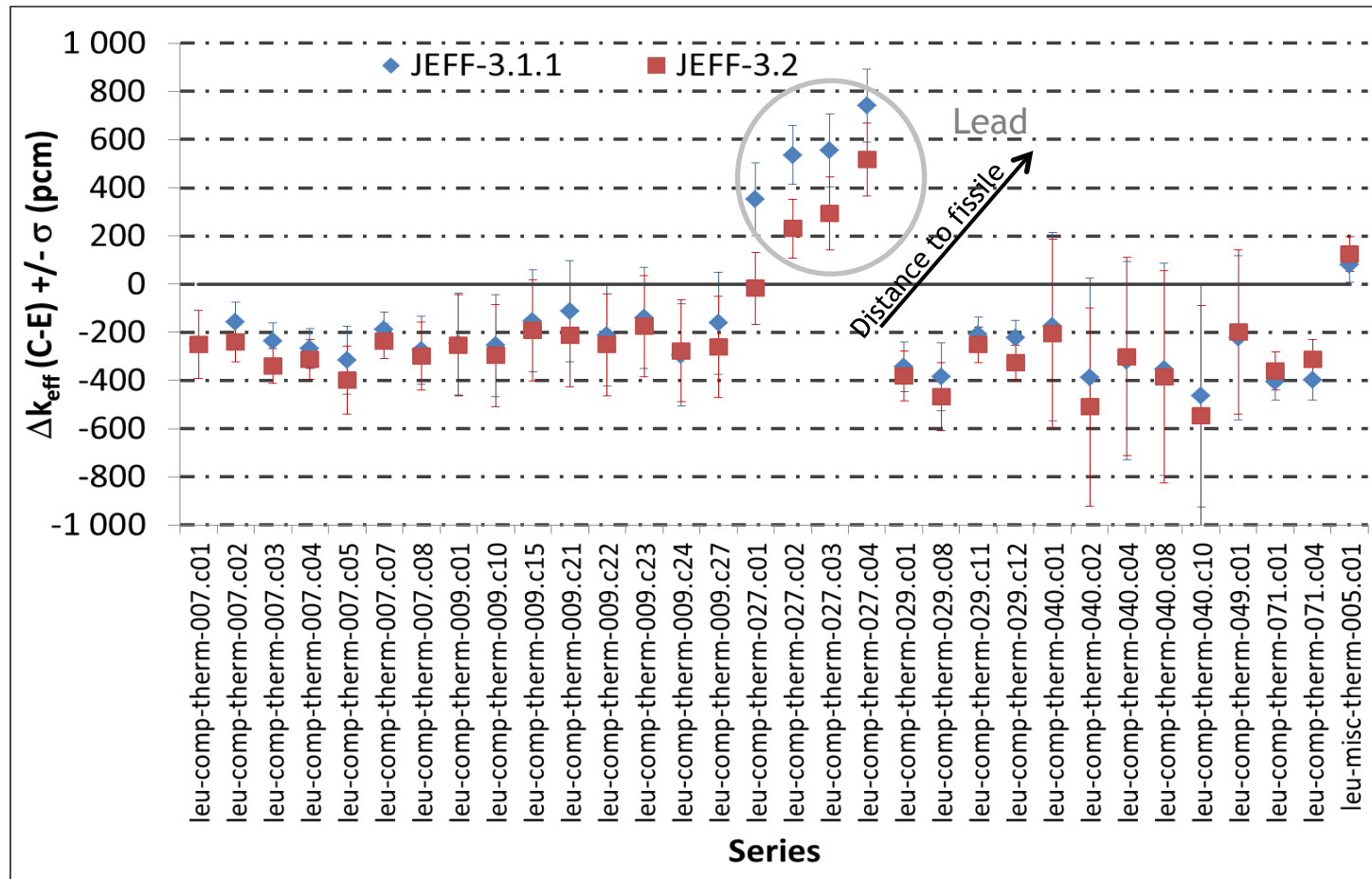
↗ Intermediate and thermal energy ranges: increase by +1000 pcm (^{232}Th)

THERMAL energy range (HEU-SOL)



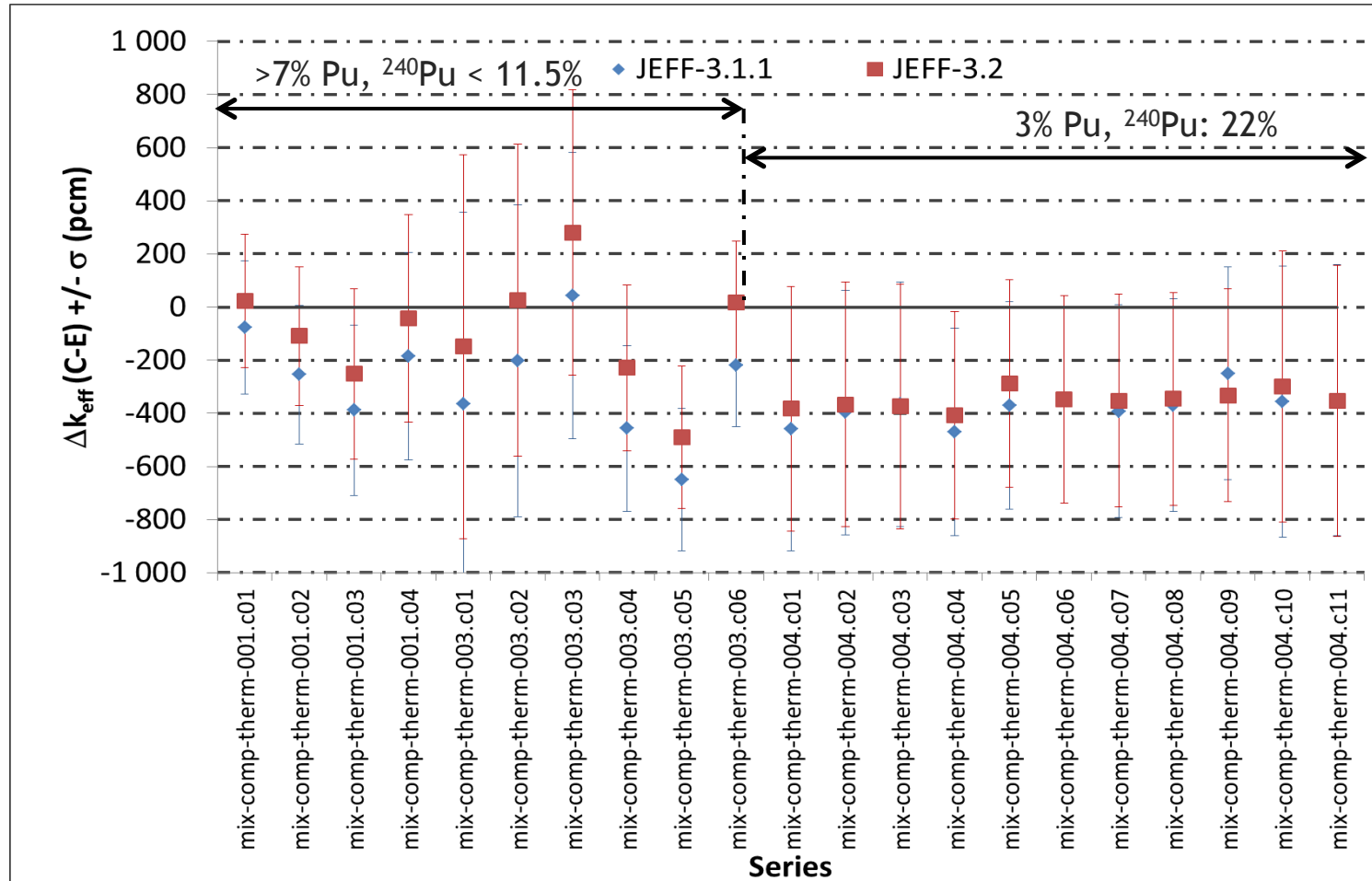
- ↗ Light water: no significant modification of k_{eff} results
- ↗ HST-004 and -020 with heavy water: increase of k_{eff} between +500 and +1000 pcm
- ↗ HST-046 involving beryllium reflector: no improvement of results

THERMAL energy range (LEU-COMP)



- ↖ Lattices in light water, without canister: no significant modification of k_{eff} results
- ↖ LCT-027: decrease of k_{eff} results due to lead new evaluation BUT improvement needed (tendency with distance of lead reflector from fissile)

THERMAL energy range (MIX-COMP)



- ↗ Improvement of results for high Pu content (MCT-001 and -003)
- ↗ No improvement for MCT-004, results in the uncertainty margins

Other Issues

Case	Benchmark k_{eff} (Unc. 1 σ)	k_{eff}			
		JEFF-3.1.1	JEFF-3.2		
		All isotopes	All isotopes	Only isotope of interest (JEFF-3.1.1 for others)	
IMF-007.c01 (BIG TEN)	1.00480 (0.00020)	0.99783	1.00359	^{238}U	1.00429
				^{235}U	1.00022
				$^{238}\text{U} + ^{235}\text{U}$	1.00536
HCT-021.c01	1.00080 (0.00290)	0.99280	0.99680	^{232}Th	0.99558
U233-MF-001	1.00000 (0.00100)	1.00639	0.99996	^{233}U	0.99986

➤ BIG TEN (reflection by DU): increase of k_{eff} due to ^{235}U and ^{238}U

➤ Improvement of results

➤ HCT-021: improvement of results with new ^{232}Th (within 2 σ)

➤ U233-MF-001: decrease of k_{eff} by -600 pcm with JEFF-3.2, improvement of results

Conclusions

■ Thermal energy spectrum

- No significant improvement for Pu, LEU and HEU solutions
- Main improvement due to:
 - The treatment of lead as absorbing elements (canisters)
 - Under-estimation with thorium partly compensated

■ Intermediate energy spectrum

- Increase of k_{eff} for systems with HEU

■ Fast energy spectrum

- Pu: no modification
- HEU: no more under-estimation with JEFF-3.2
- Decrease of k_{eff} for systems with ^{232}Th and W reflectors: better agreement with benchmark BUT improvement still needed

Conclusions (continued)

■ Fast energy spectrum

- ^{233}U : decrease of k_{eff} by -600 pcm
- Good agreement with benchmark

■ Analysis based on a restricted number of experiments

- Focus on few isotopes
 - Main tendencies highlighted
- Need to go more deeply to explain all the results (compensation between various isotopes)

■ To be done:

- Use of sensitivity coefficients to better understand trends

Thanks for your attention!!

Questions ??

The MORET 5 code

Two calculation routes

- Multi-group (281 groups) in APOLLO2-MORET 5 route (CRISTAL package)
- Continuous energy

Continuous energy 5.C.1 version using various libraries and probability tables for URR

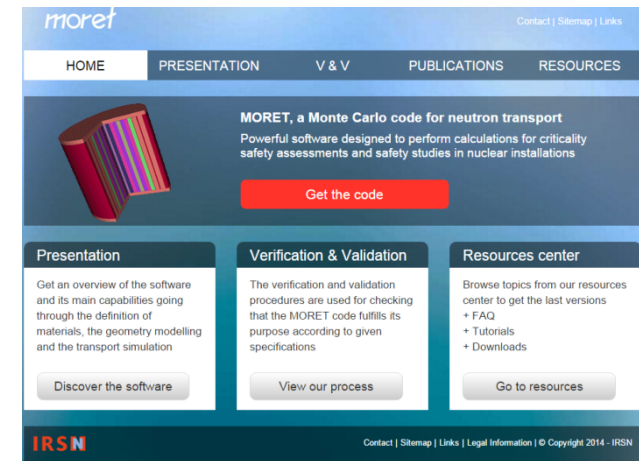
- ACE formatted files for nuclear data library: JEFF-3.1 in this study

Various simulation methods available (default: natural)

Modular geometry

Sensitivity calculation module

Correlated sampling method for chemical perturbations



<http://moret.irsn.fr/>